

# Status Report - FMR (United States of America)

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## 1. Overview

- ☐ In Operation
- ☐ Under Construction
- ☐ Described in

Link to PRIS --

☒ **New Concept**

Number of Units --

Year 2030

### Reference Plant

Name of Site --

Link to PRIS --

Net Power Output (MWe) 42

Gross Power Output (MWe) --

### 1.1 Introduction

The Fast Modular Reactor (FMR) is a 100-MW thermal Gas-cooled Fast Reactor (GFR) being developed by General Atomics Electromagnetic Systems (GA-EMS) as a flexible and dispatchable power source to the US electricity market in the mid-2030s. The goal is to develop a safe, maintainable, cost-effective, and distributed nuclear energy-generating station based on simplified design features such as an inert coolant material, pellet-loaded fuel rods, installations with air-cooling as ultimate heat sink, and small and passive heat removal systems. Specific technologies selected to achieve this goal are helium coolant that is an inert gas with no chemical reaction with structural components, not activated, single phase, enabling high temperature operation and high thermal efficiency Brayton cycle; conventional uranium dioxide fuel, which is the most widely used and well-known fuel material, capable of high burnup and long fuel life; and silicon carbide (SiC) composite (SiGA®) cladding and internal structures, which are chemically inert in the helium environment and exceptionally radiation tolerant.

### 1.2 Development Milestones

|      |                                                                 |
|------|-----------------------------------------------------------------|
| 2021 | Start of design                                                 |
| 2026 | Concept design complete                                         |
| 2027 | Start of pre-licensing vendor design review (in USA)            |
| 2029 | Engineering design complete/ Secure necessary licenses (in USA) |
| 2030 | Start construction of a demonstration unit (in USA)             |
| 2036 | Commercial operation                                            |

### 1.3 Design organization or vendor company

Christina.Back@ga.com

### 1.4 Links to designer/vendor website

<https://www.ga.com/>

### 1.5 Design Description



Figure 1. Reactor building and major systems and components arrangement designed with defense-in-depth.

### 1.6 Licensing Application Support Documents

Most Recent Licensing Application Support Document:

- Principal Design Criteria (PDC)
- Quality Assurance Program (QAP)

## 1.7 Reactor Unit in PRIS

## 2. Category Fields

Table 1: ARIS Category Fields

| ARIS Category                                  | Value                                | Units of Examples                                                                                                                   |
|------------------------------------------------|--------------------------------------|-------------------------------------------------------------------------------------------------------------------------------------|
| Current/Intended Purpose                       | Commercial – Electric/Non-Electric   | e.g. Commercial – Electric/Non-electric, Prototype/FOAK, Demonstration, Experimental                                                |
| Main Intended Application (once commercial)    | Dispatchable                         | e.g. Baseload, Dispatchable, Off-grid/Remote, Mobile/Propulsion, Non-electric (specify)                                             |
| Reference Location                             | Below-Ground                         | e.g. On Coast, Inland, Below-Ground, Floating-Fixed, Marine-Mobile, Submerged-Fixed (Other-specify)                                 |
| Reference Site Design (reactor units per site) | Multiple Units                       | e.g. Single Unit, Dual Unit, Multiple Unit (# units)                                                                                |
| Core Coolant                                   | He                                   | H <sub>2</sub> O, D <sub>2</sub> O, He, CO <sub>2</sub> , Na, Pb, PbBi, Molten Salts, (Other-specify)                               |
| Reactor Core Size (1 core)                     | 30 MWth <= Thermal power < 1000 MWth | e.g. Thermal Power < 30 MWth, 30 MWth ≤ Thermal power < 1000 MWth, 1000 MWth ≤ Thermal power ≤ 3000 MWth, Thermal power > 3000 MWth |
| Reactor Type                                   | GFR                                  | e.g. PWR, BWR, HWR, SCWR, GCR, GFR, SFR, LFR, MSR, ADS                                                                              |
| Reactor Size                                   | SMR                                  | e.g. PWR, SMR                                                                                                                       |
| Neutron Moderator                              | None                                 | e.g. H <sub>2</sub> O, D <sub>2</sub> O, He, CO <sub>2</sub> , Na, Pb, PbBi, Molten Salts, (Other-specify)                          |
| NSSS Layout                                    | Direct-cycle                         | e.g. Loop-type (# loops), Direct-cycle, Semi-integral, Integral, Pool-type                                                          |

|                                    |                      |                                                                                                               |
|------------------------------------|----------------------|---------------------------------------------------------------------------------------------------------------|
| Primary Circulation                | Forced (1 pumps)     | e.g. Forced (# pumps), Natural                                                                                |
| Thermodynamic Cycle                | Brayton              | e.g. Rankine, Brayton, Combined-Cycle (direct/indirect)                                                       |
| Secondary Side Fluid               | Other                | e.g. H <sub>2</sub> O, He, CO <sub>2</sub> , Na, Pb, PbBi, Molten Salts, (Other-specify)                      |
| Fuel Form                          | Fuel Assembly/Bundle | e.g. Fuel Assembly/Bundle, Coated Sphere, Plate, Prismatic, Contained Liquid, Liquid Fuel/Coolant             |
| Fuel Lattice Shape                 | Triangular           | e.g. Square, Hexagonal, Triangular, Cylindrical, Spherical, Other, n/a                                        |
| Rods/Pins per Fuel Assembly/Bundle | 120                  | e.g. #, n/a                                                                                                   |
| Fuel Material Type                 | Oxide                | e.g. Oxide, Nitride, Carbide, Metal, Molten Salt, (Other-specify)                                             |
| Design Status                      | Conceptual Design    | e.g. Conceptual, Detailed, Final (with secure suppliers)                                                      |
| Licensing Status                   |                      | e.g. DCR, GDR, PSAR, FSAR, Design Licensed (in Country), Under Construction (# units), In Operation (# units) |
| Neutron Spectrum                   | Fast                 | e.g. Fast, Thermal                                                                                            |

### 3. Parameter Fields

Table 2: ARIS Parameter Fields

| ARIS Parameter                             | Value | Units of Examples                                                                             |
|--------------------------------------------|-------|-----------------------------------------------------------------------------------------------|
| <b>Plant Infrastructure</b>                |       |                                                                                               |
| Design Life                                | 60    | Years                                                                                         |
| Lifetime Capacity Factor                   | 90    | %, defined as Lifetime MWe-<br>yrs delivered / (MWe capacity<br>* Design Life), incl. outages |
| Major Planned Outages                      | 22    | # days every # months<br>(specify purpose, including<br>refuelling)                           |
| Operation / Maintenance<br>Human Resources | 10/25 | # Staff in Operation /<br>Maintenance Crew during<br>Normal Operation                         |

|                                                  |                           |                                                                                 |
|--------------------------------------------------|---------------------------|---------------------------------------------------------------------------------|
| Reference Site Design                            | 1                         | n Units/Modules                                                                 |
| Capacity to Electric Grid                        | 42                        | MWe (net to grid)                                                               |
| Non-electric Capacity                            | N/A                       | e.g. MW(th) heat at x °C,<br>m3/day desalinated water,<br>kg/day hydrogen, etc. |
| In-House Plant Consumption                       | 1.9                       | MWe                                                                             |
| Plant Footprint                                  | 2000                      | m2 (rectangular building<br>envelope)                                           |
| Site Footprint                                   | 38000                     | m2 (fenced area)                                                                |
| Emergency Planning Zone                          |                           | km (radius)                                                                     |
| Releases during Normal<br>Operation              | 0/0/0                     | TBq/yr (Noble Gases / Tritium<br>Gas / Liquids)                                 |
| Load Following Range and<br>Speed                | 50 – 100%, 20% per minute | x – 100%, % per minute                                                          |
| Seismic Design (SSE)                             | 0.3                       | g (Safe-Shutdown<br>Earthquake)                                                 |
| NSSS Operating Pressure<br>(primary side)        | 7                         | MPa(abs), i.e. MPa(g)+0.1, at<br>core outlet                                    |
| NSSS Operating Pressure<br>(secondary side)      |                           | MPa(abs), i.e. MPa(g)+0.1, at<br>secondary side outlet                          |
| Primary Coolant Inventory<br>(incl. pressurizer) | 210                       | kg                                                                              |
| Nominal Coolant Flow Rate<br>(primary)           | 67                        | kg/s                                                                            |
| Nominal Coolant Flow Rate<br>(secondary)         |                           | kg/s                                                                            |
| Core Inlet Coolant<br>Temperature                | 506                       | °C                                                                              |
| Core Outlet Coolant<br>Temperature               | 800                       | °C                                                                              |
| Available Temperature as<br>Process Heat Source  | N/A                       | °C                                                                              |
| NSSS Largest Component                           | RPV                       | e.g. RPV (empty), SG, Core<br>Module (empty/fuelled), etc.                      |
| - NSSS Largest Component<br>Length               | 11000                     | mm                                                                              |
| - NSSS Largest Component<br>Diameter             | 4600                      | mm                                                                              |

|                                                |                            |                                                                        |
|------------------------------------------------|----------------------------|------------------------------------------------------------------------|
| - NSSS Largest Component Thickness             |                            | mm                                                                     |
| - NSSS Largest Component Weight                | 230                        | tonne                                                                  |
| Reactor Vessel Material                        | 316H                       | e.g. SS304, SS316, SA508, 800H, Hastelloy N                            |
| Steam Generator Design                         | N/A                        | e.g. Vertical/Horizontal, U-Tube/ Straight/Helical, cross/counter flow |
| Secondary Coolant Inventory                    | N/A                        | kg                                                                     |
| Pressurizer Design                             | None                       | e.g. separate vessel, integral, steam or gas pressurised, etc.         |
| Pressurizer Volume                             | N/A                        | m <sup>3</sup> / m <sup>3</sup> (total / liquid)                       |
| Containment Type                               | Dry (single)               | Dry (single/double), Dry/Wet Well, Inerted, etc.                       |
| Containment Volume                             | 4500                       | m <sup>3</sup>                                                         |
| Spent Fuel Pool Capacity and Total Volume      | 60/2100                    | Years of full-power operation / m <sup>3</sup>                         |
| Non Electric Application                       |                            | -                                                                      |
| Core Delta Temperature                         | 294                        | °C                                                                     |
| <b>Fuel/Core</b>                               |                            |                                                                        |
| Single Core Thermal Power                      | 100                        | MW(th)                                                                 |
| Refuelling Cycle                               | 180                        | months or “continuous”                                                 |
| Fuel Material                                  | UO <sub>2</sub>            | e.g. UO <sub>2</sub> , MOX, UF <sub>4</sub> , UCO                      |
| Enrichment (avg./max.)                         | 17.6/19.75                 | %                                                                      |
| Average Neutron Energy                         | 100000                     | eV                                                                     |
| Fuel Cladding Material                         | SiC                        | e.g. Zr-4, SS, TRISO, E-110, none                                      |
| Number of Fuel “Units”                         | 198                        | Specify as Assembly, Bundle, Plate, Sphere, or n/a                     |
| Weight of one Fuel Unit                        | 140                        | kg                                                                     |
| Total Fissile Loading (initial)                | 3100/U-235/UO <sub>2</sub> | kg fissile material (specify isotopic and chemical composition)        |
| % of fuel outside core during normal operation | N/A                        | Applicable to online refuelling and molten salt reactors               |

|                                                              |                  |                                                                                      |
|--------------------------------------------------------------|------------------|--------------------------------------------------------------------------------------|
| Fraction of fresh-fuel fissile material used up at discharge | 47               | %                                                                                    |
| Core Discharge Burnup                                        | 100              | MWd/kgHM (heavy metal, eg U, Pu, Th)                                                 |
| Pin Burnup (max.)                                            | 135              | MWd/kgHM                                                                             |
| Breeding/Conversion Ratio                                    | <1               | Fraction of fissile material bred in-situ over one fuel cycle or at equilibrium core |
| Reprocessing                                                 | None             | e.g. None, Batch, Continuous (FP polishing/actinide removal), etc.                   |
| Main Reactivity Control                                      | Rods             | e.g. Rods, Boron Solution, Fuel Load, Temperature, Flow Rate, Reflectors             |
| Solid Burnable Absorber                                      | B4C              | e.g. Gd2O3                                                                           |
| Core Volume (active)                                         | 7.5              | m3 (used to calculate power density)                                                 |
| Fast Neutron Flux at Core Pressure Boundary                  | 7E9              | n/(cm <sup>2</sup> *s)                                                               |
| Max. Fast Neutron Flux                                       | 1.3E14           | n/(cm <sup>2</sup> *s)                                                               |
| Core Control Rod Absorber                                    |                  | -                                                                                    |
| Core Soluble Neutron Absorber                                |                  | -                                                                                    |
| <b>Safety Systems</b>                                        |                  |                                                                                      |
| Number of Safety Trains                                      | Active / Passive | % capacity of each train to fulfil safety function                                   |
| - reactor shutdown                                           | Active/Passive   | 100 / 100 / -                                                                        |
| - core injection                                             | Active/Passive   | 100 / - / -                                                                          |
| - decay heat removal                                         | Active/Passive   | 100 / 100 / -                                                                        |
| - containment isolation and cooling                          | Active/Passive   | 100 / - / -                                                                          |
| - emergency AC supply (e.g. diesels)                         | Active/Passive   | - / - / -                                                                            |
| DC Power Capacity (e.g. batteries)                           |                  | Hours                                                                                |
| Events in which Immediate Operator Action is required        | None             | e.g. any internal/external initiating events, none                                   |

|                                                                                                                             |               |                                                                                                         |
|-----------------------------------------------------------------------------------------------------------------------------|---------------|---------------------------------------------------------------------------------------------------------|
| Limiting (shortest) Subsequent Operator Action Time                                                                         |               | Hours (that are assumed when following EOPs)                                                            |
| Severe Accident Core Provisions                                                                                             | No core melt  | e.g. no core melt, IMMR, Core Catcher, Core Dump Tank, MCCI                                             |
| Core Damage Frequency (CDF)                                                                                                 |               | x/reactor-year (based on reference site and location)                                                   |
| Severe Accident Containment Provisions                                                                                      | None          | e.g., H2 ignitors, PARs, filtered venting etc.                                                          |
| Large Release Frequency (LRF)                                                                                               | 0             | x/reactor-year (based on reference site and location)                                                   |
| <b>Overall Build Project Costs Estimate or Range (excluding Licensing, based on the Reference Design Site and Location)</b> |               |                                                                                                         |
| Construction Time (nth of a kind)                                                                                           | 24            | Months from first concrete to criticality                                                               |
| Design, Project Mgmt. and Procurement Effort                                                                                |               | Person-years (PY) [DP&P]                                                                                |
| Construction and Commissioning Effort                                                                                       |               | PY [C&C]                                                                                                |
| Material and Equipment Overnight Capital Cost                                                                               |               | Million US\$(2015) [M&E], if built in USA                                                               |
| Cost Breakdown                                                                                                              | %[C&C]/%[M&E] |                                                                                                         |
| - Site Development before first concrete                                                                                    |               | (e.g. 25/10 )<br>( 30/40 )<br>( 20/25 )<br>( 20/10 )<br>( 5/15 )<br>( ----- )<br>(to add up to 100/100) |
| - Nuclear Island (NSSS)                                                                                                     |               |                                                                                                         |
| - Conventional Island (Turbine and Cooling)                                                                                 |               |                                                                                                         |
| - Balance of Plant (BOP)                                                                                                    |               |                                                                                                         |
| - Commissioning and First Fuel Loading                                                                                      |               |                                                                                                         |
| Factory / On-Site split in [C&C] effort                                                                                     |               | %/% of total [C&C] effort in PY (e.g., 60/40 )                                                          |
| <b>Nuclear System</b>                                                                                                       |               |                                                                                                         |
| System Steam Flow Rate                                                                                                      |               | kg/s                                                                                                    |
| System Steam Pressure                                                                                                       |               | MPa                                                                                                     |
| System Steam Temperature                                                                                                    |               | °C                                                                                                      |
| System Feedwater Flow Rate                                                                                                  |               | kg/s                                                                                                    |



|                              |       |                   |
|------------------------------|-------|-------------------|
| System Feedwater Temperature |       | °C                |
| <b>Reactor Core</b>          |       |                   |
| Active Core Height           |       | m                 |
| Equivalent Core Diameter     |       | m                 |
| Average Linear Heat Rate     |       | kW/m              |
| Average Fuel Power Density   |       | kW/kgU            |
| Average Core Power Density   | 13.33 | MW/m <sup>3</sup> |
| Outer Diameter Of Fuel Rods  |       | mm                |

## 4. Plant Layout, Site Environment and Grid Integration

### SUMMARY FOR BOOKLET

The FMR plant contains five main buildings for the reactor, auxiliary system, cooling tower, warehouse, and administration. The reactor building includes the underground containment structure, which contains the sealed containment vessel, reactor, power conversion system, and reactor cooling system, as well as the reactor vessel passive cooling system, control room, spent fuel storage, helium service system, and support equipment. The reference design allocates 9.5 acres for the FMR plant, excluding the switchyard and exclusion zones. The total space allocation including the switchyard is 16.5 acres.



Figure 2. Site-flexible FMR plant layout.

#### 4.1 Site Requirements during Construction

#### 4.2 Site Considerations during Operation

#### 4.3 Grid Integration

## 5. Technical NSSS/Power Conversion System Design

### SUMMARY FOR BOOKLET

#### **Primary circuit**

The primary circuit consists of reactor system coupled in fluid communication to the power conversion unit. The reactor pressure vessel and power conversion unit are coupled by a cross-duct that comprises concentric inner and outer flow passageways. The power conversion unit includes a turbine, compressors, a generator, and heat exchangers. The fluid-driven turbine-generator provides a continuous output of electrical power. The reactor core cooling is conducted by the power conversion unit during normal operation and shutdown.

#### **Reactor core and Fuel**

The reactor system has a compact annular core surrounded by advanced fast neutron reflector material, i.e., zirconium silicide, graphite, and steel vessel. Low core power density and use of uranium dioxide fuel and silicon carbide composite matrix ceramic cladding secures the safety margin of the fuel. A three-batch refuelling scheme is used to enhance higher fuel utilization and reduce the requirement of excess reactivity control. The average discharge burnup is 100 GWd/tU with high assay low enriched uranium.

#### **Fuel handling**

Fuel handling activities are carried out from the floor of the maintenance hall. All access to and removal of the fuel will be by remote handling. The refuelling equipment includes a dry spent fuel storage canister and a fuel-handling machine.

#### **Reactor protection**

The reactor protection system provides automatic protective actions. These actions are accomplished by detecting abnormal plant conditions and initiating safety protective action for any condition that could threaten the integrity of a fission product barrier and, consequently, the public safety. The protection system works in functional groups that provide separation for the two types of reactor trips and all engineered safeguards.

#### **Secondary side**

The turbo-compressor is designed at 800 °C and 7 MPa, and for a 44-MW electrical energy output. The permanent magnet generator and variable frequency converter generate electricity. The generator operates with variable turbine speeds from 6,000 rpm to 11,400 rpm and can operate as a pump motor for helium circulation during startup operations. The reference design is at 60 Hz, but it can be configured to 50 Hz. This heat sink is dry cooling with cooling towers sited outside the reactor building.

#### **Containment**

The sealed containment vessel is a free-standing cylindrical pressure vessel constructed of stainless-

steel plate that is located inside the concrete containment building, independent of the concrete mat foundation, walls, or slab above. The main function of the sealed containment is to prevent the uncontrolled release of radiological material into the surrounding environment under normal operating, abnormal accident, or extreme environmental conditions including severe accident temperature and pressure changes.

### **Electrical, I&C and human interface**

The plant electrical system includes the AC/DC power distribution, grounding and lightning protection, backup electric supply, and plant uninterruptible power supply. The I&C consists of the safety-related reactor protection system (RPS) and the non-safety-related plant protection system (PPS), plant control, data, and instrumentation system (PCDIS), and plant monitoring system (PMS). The I&C employs methods such as prioritization and plant state dependency for alarms, to avoid operator information overload caused by untimely or unnecessary audible or visual information.

### **Unique technical design features**

The unique design features of the FMR include the helium coolant that provides super operational safety due to its single phase, non-radioactive, non-corrosive, non-toxic, and optically transparent properties; use of SiC composite material that enables high-temperature and high-burnup operation for high thermal efficiency and passive safety; dry-cooling heat sink that enables siting and quick installation at geographically diverse nodes and off-grid locations, without need for a large water source; and closed Brayton cycle that provides a high efficiency, resulting in economic benefit as well as reduced fuel consumption and high-level waste.

## **5.1 Primary Circuit**

The primary circuit consists of reactor system coupled in fluid communication to the power conversion unit. The reactor pressure vessel and power conversion unit are coupled by a cross-duct that comprises concentric inner and outer flow passageways. The power conversion unit includes a turbine, compressors, a generator, and heat exchangers. The fluid-driven turbine-generator provides a continuous output of electrical power. The reactor core cooling is conducted by the power conversion unit during normal operation and shutdown.

## **5.2 Reactor Core and Fuel**

The reactor system has a compact annular core surrounded by advanced fast neutron reflector material, i.e., zirconium silicide, graphite, and steel vessel. Low core power density and use of uranium dioxide fuel and silicon carbide composite matrix ceramic cladding secures the safety margin of the fuel. A three-batch refuelling scheme is used to enhance higher fuel utilization and reduce the requirement of excess reactivity control. The average discharge burnup is 100 GWd/tU with high assay low enriched uranium.

## **5.3 Fuel Handling**

Fuel handling activities are carried out from the floor of the maintenance hall. All access to and removal of the fuel will be by remote handling. The refuelling equipment includes a dry spent fuel storage canister and a fuel-handling machine.

## **5.4 Reactor Protection**

The reactor protection system provides automatic protective actions. These actions are accomplished by detecting abnormal plant conditions and initiating safety protective action for any condition that could threaten the integrity of a fission product barrier and, consequently, the public safety. The protection system works in functional groups that provide separation for the two types of reactor trips and all engineered safeguards.

## **5.5 Secondary Side**

The turbo-compressor is designed at 800 °C and 7 MPa, and for a 44-MW electrical energy output. The permanent magnet generator and variable frequency converter generate electricity. The generator operates with variable turbine speeds from 6,000 rpm to 11,400 rpm and can operate as a pump motor for helium circulation during startup operations. The reference design is at 60 Hz, but it can be configured to 50 Hz. This heat sink is dry cooling with cooling towers sited outside the reactor building.

## **5.6 Containment/Confinement**

The sealed containment vessel is a free-standing cylindrical pressure vessel constructed of stainless-steel plate that is located inside the concrete containment building, independent of the concrete mat foundation, walls, or slab above. The main function of the sealed containment is to prevent the uncontrolled release of radiological material into the surrounding environment under normal operating, abnormal accident, or extreme environmental conditions including severe accident temperature and pressure changes.

## **5.7 Electrical, I&C and Human Interface**

The plant electrical system includes the AC/DC power distribution, grounding and lightning protection, backup electric supply, and plant uninterruptible power supply. The I&C consists of the safety-related reactor protection system (RPS) and the non-safety-related plant protection system (PPS), plant control, data, and instrumentation system (PCDIS), and plant monitoring system (PMS). The I&C employs methods such as prioritization and plant state dependency for alarms, to avoid operator information overload caused by untimely or unnecessary audible or visual information.

## **5.8 Unique Technical Design Features (if any)**

The unique design features of the FMR include the helium coolant that provides super operational safety due to its single phase, non-radioactive, non-corrosive, non-toxic, and optically transparent properties; use of SiC composite material that enables high-temperature and high-burnup operation for high thermal efficiency and passive safety; dry-cooling heat sink that enables siting and quick installation at geographically diverse nodes and off-grid locations, without need for a large water source; and closed Brayton cycle that provides a high efficiency, resulting in economic benefit as well as reduced fuel consumption and high-level waste.

# **6. Technology Readiness**

## **SUMMARY FOR BOOKLET**

There are key systems and components specific to the FMR that need to be developed fully for a design to achieve its goal. Examples are the fully ceramic fuel element, reactor enclosure for the

helium system, and safety system that require experimental verification of the design. The technology readiness level of the FMR is at 4 as the experimental measurements have been partially conducted for the high-burnup UO<sub>2</sub> fuel measurements of fission gas release and swelling, a series of thermal, mechanical, and safety measurements of the SiC composite tube, low-fluence irradiation of zirconium silicide reflector, and passive reactor vessel cooling.

### **6.1 Deployed Reactors**

### **6.2 Reactors under Licensing Review**

### **6.3 Reactors in the Design Stage**

## **7. Safety Concepts**

### **SUMMARY FOR BOOKLET**

The FMR safety is enhanced by materials and design features. The helium coolant is non-hazardous, non-explosive, non-corrosive, and not activated. The annular core configuration reduces the power peaking but promotes cooling capability during emergency. Low power density and high-temperature materials secure the safety margin in the highly unlikely event of an accident. The engineered safety features (ESFs) of the FMR mitigate the consequences of postulated accidents. The ESFs include reactivity control system, reactor protection system, maintenance cooling system, reactor vessel cooling system (RVCS), and containment. The RVCS provides a redundant, passive means of transporting core residual heat during the loss of coolant accident (LOCA) followed by station blackout. The heat is removed by radiation from the reactor vessel and by natural circulation of water through the cooling system, keeping the reactor core and vessel temperatures under the design limits.

### **7.1 Safety Philosophy and Implementation**

### **7.2 Transient/Accident Behaviour**

## **8. Fuel and Fuel Cycle**

### **SUMMARY FOR BOOKLET**

The reference FMR design is based on once-through fuel cycle with interim storage and direct disposal. Recycling is feasible through a proliferation-resistant dry process that uses only thermal-mechanical treatment of oxide fuel without any involvement of chemicals.

The radioactive waste management systems segregate, collect, process, store, and dispose of radioactive liquid, gaseous, and solid wastes. The liquid waste system includes filtration, demineralization, and discharging processed wastes to the solid waste system. The gaseous waste system reduces the activity of the waste gas to a level acceptable for discharge to the atmosphere.

The solid waste systems include collection, solidification, packaging, and storage facilities for radioactive materials prior to their offsite shipment for disposal.

### **8.1 Fuel Cycle Options**

### **8.2 Resource Use Optimization**

### **8.3 Unique Fuel/Fuel Cycle Design Features**

## **9. Safeguard & Physical Security**

### **SUMMARY FOR BOOKLET**

The FMR has no new or unique proliferation risks. For example, individual fuel assemblies are uniquely identified for item counting, and there is no on-line refuelling or liquid fuel. The fuelling interval is 15-year, and the total discharge is two full cores after 60-year life.

Transparent gas cooling does not interfere with inspection. Spent fuel storage is secure, below grade with restricted access; therefore, surveillance of access floor is sufficient for domestic and IAEA requirements. Spent fuels remain in storage area throughout plant lifetime and are shipped off-site during decommissioning. The plant design incorporates measures to prevent unauthorized access and/or theft of nuclear material, complying with 10 CFR 73.55 for physical protection of licensed activities in nuclear power reactors against radiological sabotage and 10 CFR 73.67 for the physical protection of special nuclear material of moderate and low strategic significance.

### **9.1 Safeguards**

### **9.2 Security**

### **9.3 Unique Safeguards and/or Security Features**

## **10. Project Delivery & Economics**

### **SUMMARY FOR BOOKLET**

The FMR will impact the distributed energy market by providing a safe, carbon-free source of energy that is compatible with renewables and is economically viable with modular design, factory-fabrication, truck-transportation, on-site assembly, site flexibility, and low cost of capital features. The FMR is important for the 2035-and-beyond needs of the world to serve the electric power customers such as those in unique areas with almost no grid support, independent system operators (ISOs) in need of small, distributed generation, and capacity markets suffering from the continuing retirement of conventional baseload generation. The LCOE of the Nth-of-a-kind (NOAK) FMR is targeted to be \$70/MWh or less.

**10.1 Project Preparation and Negotiation**

**10.2 Construction and Commissioning**

**10.3 Operation and Maintenance**